

# IAEA/ANL Interregional Training Course



# Technical and Administrative Preparations Required for Shipment of Research Reactor Spent Fuel to Its Country of Origin

Argonne National Laboratory Argonne, IL 13 - 24 January 1997

#### Lecture L.3.1a

## Savannah River Site Appendix A Agreement Preparation Guidelines

Sample Appendix A for Generic MTR Assembly

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# APPENDIX A AGREEMENT SPENT NUCLEAR FUEL ACCEPTANCE CRITERIA

No WITH _		CONTRACT N	0
THIS AGREEMENT, entered into this	day of	, 19	, constitutes an agreement by
the U.S. Department of Energy (hereinafter	called DOE) to	receive under th	e terms and conditions of Contra
No, the specific	ation material de	escribed herein.	This agreement provides a detailed
description of the material to be delivered to	DOE in accord	ance with this co	ontract and also enumerates the
specifications and requirements which the C	Customer must m	eet. Failure of t	he material delivered hereunder to
comply with the specifications and requiren	nents given in thi	s agreement wil	l result in the material being non-
specification material. A separate Appendix	A Agreement w	ill be required for	or each element or assembly which
is different in Description. All dimensions i	must be given in	centimeters and	all weights in grams.

#### A. Correspondence

#### 1. Customer Contact

Laboratory/ Research Center/University	Savannah River Site
Reactor Name	Material Test Reactor
City, State, Country	Aiken, SC, USA
<b>Customer Name</b>	Trent C. Andes
<b>Customer Signature</b>	
Title	Reactor Manager
Phone Number	803-557-9483
Fax Number	803-557-9080
Date	06 January 1997

#### 2. Department of Energy Contact

All correspondence / inquiries regarding this document and the information contained herein shall be directed to:

U.S. Department of Energy Savannah River Operations Office Spent Fuel Storage Division P.O. Box A Aiken, SC 29801

Phone and facsimile inquiries may be made to: Phone: (803)-557-3759

Fax: (803)-557-3763

#### B. Definitions

The following definitions are applied to the Specification Material described in this Agreement:

**Fuel Element** -The smallest integral unit of clad fuel containing source or special nuclear material (e.g., plate, tube, rod, disc, etc.).

**Assembly -** A group of elements combined in a structural unit. The assembly is usually that fuel structure which is removed from the reactor as an individual unit.

#### C. Form and Composition of Specification Material

#### 1. Drawing Identification

The following design or fuel fabrication drawing(s), six (6) copies of which are attached and which are incorporated herein by reference thereto, constitute(s) a comprehensive and accurate illustration of the fuel elements and assemblies to be delivered under this Contract.

Drawing No. / Rev. No. / English Title
5001 / A / 19 Plate MTR Assembly
5002 / A / Type 8 Fuel Plate (Flat)
5003 / A / Type 8 Fuel Plate (Formed)
5004 / A / Type 9 Fuel Plate (Flat)
5005 / A / Type 9 Fuel Plate (Formed)
5006 / A / Fuel Compact - Enriched, Type 8 and Type 9
5007 /A / Side Plate
5008 / A / End Box

Note: If fuel is cropped (cut), drawings should indicate the location of the cut(s).

#### 2. Material Description

The following tables summarize the description of fuel elements and assemblies to be delivered under this contract. Where dimensions are required, the nominal dimensions from the fuel element and assembly drawings must be used. If changes in dimensions have occurred due to cropping or other modification, the best estimate of the maximum change in these dimensions must be given. Weights must be dry, unirradiated weights . Where isotopic weights of SNM are required, tolerances shall be specified.

**(a.1) Fuel 'Element' Description** (If more than one type of element per assembly, divide the space to describe each type of element or duplicate this page as necessary.)

Fuel element type (curved or flat plate,	curved plate (Type 9, outer)
tube, rod, disc, etc.)	
Nominal dimensions (include clad & bond, cm) <sup>a</sup>	68.90 x 7.10 x 0.164
Nominal dimensions of fuel meat (cm)a, b	59.79 x 6.24 x 0.05
Nominal total weight of fuel element (g)	227.0
Chemical form of fuel meat (e.g. UAl <sub>x</sub> -alloy, U3O <sub>8</sub> -Al, U3Si <sub>2</sub> -Al, etc.)	U <sub>3</sub> O <sub>8</sub> in Aluminum matrix
Nominal total weight of fuel meat (g)	62.0
Weight of total $U$ ( $g \pm g$ uncertainty)	$12.45 \pm 0.27$
Weight of U-235 ( $g \pm g$ uncertainty)	$11.58 \pm 0.25$
Matrix material, weight (g)	101 Al powder; 47.0
Cladding material & method of sealing	6061 Al; roll-bonded
Clad thickness (cm), total clad weight (g)	0.057; 165.0
Bonding material, if any (Na, Al-Si, etc.)	N/A
Bond thickness (cm), weight (g)	N/A
Other materials contained in the fuel element: (include dimensions and weights)	N/A

<sup>(</sup>a) For curved plate type element, state dimensions for flat form.

<sup>(</sup>b) As formed in element.

(a.2) Fuel 'Element' Description (If more than one type of element per assembly, divide the space to describe each type of element or duplicate this page as necessary.)

Fuel element type (curved or flat plate, tube, rod, disc, etc.)	curved plate (Type 8, inner)
Nominal dimensions (include clad & bond, cm) <sup>a</sup>	62.55 x 7.10 x 0.1275
Nominal dimensions of fuel meat (cm) <sup>a, b</sup>	59.79 x 6.24 x 0.05
Nominal total weight of fuel element (g)	164.0
Chemical form of fuel meat (e.g. UAl <sub>x</sub> -alloy, U3O <sub>8</sub> -Al <sub>1</sub> , U3Si <sub>2</sub> -Al <sub>1</sub> , etc.)	U <sub>3</sub> O <sub>8</sub> in Aluminum matrix
Nominal total weight of fuel meat (g)	62.0
Weight of total U ( $g \pm g$ uncertainty)	$12.45 \pm 0.27$
Weight of U-235 (g $\pm$ g uncertainty)	$11.58 \pm 0.25$
Matrix material, weight (g)	101 Al powder; 47.0
Cladding material & method of sealing	6061 Al; roll-bonded
Clad thickness (cm), total clad weight (g)	0.039; 102.0
Bonding material, if any (Na, Al-Si, etc.)	N/A
Bond thickness (cm), weight (g)	N/A
Other materials contained in the fuel element: (include dimensions and weights)	N/A

<sup>(</sup>a) For curved plate type element, state dimensions for flat form.

<sup>(</sup>b) As formed in element.

### (b) Fuel 'Assembly' Description

Total number of elements	19 plates (2 outer, 17 inner)
Over-all dimensions (cm) <sup>a</sup>	108.27 x 8.35 x 7.61
Over-all weight (g)	6026.8
Total weight of U (g ± g uncertainty)	236.55 ± 5.13
Total weight of U-235 (g ± g uncertainty)	220.02 ± 4.75
Enrichment (%)	93.01
Canning material (Al, SST) <sup>b</sup>	N/A
Canning dimensions (cm), weight (g)	N/A
Method of can sealing (screw, weld, etc)	N/A
Side plate material	6061-T6 Al
Side plate - quantity, dimensions (cm), weight per plate (g)	2; 68.9 x 8.05 x 0.475; 580
Side plate - outer slot depth & width (cm)	0.22; 0.18
Side plate - inner slot depth & width (cm)	0.22; 0.14
Spacer material	N/A
Spacer - quantity, dimensions (cm), weight per spacer (g)	N/A
End box material	356.0-T6 Al alloy (ASTM B-26)
End box - quantity, dimensions (cm), weight per box (g)	2; 20.88 x 7.41 x 7.24; 760.0
Braze or weld material	N/A
Braze or weld dimensions (cm), weight (g)	N/A
Other structural material in assembly (e.g. dummy plates, thermocouples, etc., include quantity, dimensions, material, & weight (g))	Pins, 8, 6mm x 8mm, Al-Mg-Si, 0.6

- (a) Has the assembly been cropped? Yes No (Circle One) If yes, indicate dimensions & weights in terms of the cropped assembly
- (b) When fuel assemblies have casings, describe using these entries.

Do the fuel elements contain Sodium (Na)? Yes No (Circle One)

#### D. Fuel Identification

Each separately removable unit in a shipment batch n	nust be identified by a durable tag or by embossing
Identification of the Units to be delivered under this A	Appendix A Agreement are as follows (NOTE:

Customer shall list actual	assembly	identification	numbers)	:
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L-006, -007, -008, -009, -010, -013, -014, -015, -016, -017, -018, -019, -020, -021	
Identification numbers are vibro-etched into two opposing sides of each assembly.	

#### E. Fuel Irradiation Specifications

1.	Fuel Irradiation History - General Summary
Irra	adiation history varies for each assembly. Post irradiation data was generated using ORIGEN.

#### 2. Post-Irradiation Specifications

The average and maximum SNM content post-irradiation is to be specified in grams per assembly. The best available value should be given and the uncertainty stated. The irradiation history for <u>each</u> assembly is to be provided according to Section F.

SNM Material	Average (g)	Maximum (g)	Uncertainty (± g or ± %)
Total U	161.40	163.21	± 5%
Total Pu	0.35	0.38	± 5%
Total Np	0.25	0.29	± 5%
Total Th			
U-232			
U-233			
U-235	132.37	135.14	± 5%
U-236	13.62	14.79	± 5%
Pu-239	0.27	0.29	± 5%
Pu-241	0.02	0.02	± 5%

#### F. Fuel Irradiation History - Assembly Specific Data

The irradiation history for <u>each</u> assembly is to be provided according to the format specified below. Additional tables may attached (if necessary).

Assembly	Pre-Irrac	diation				I			Post-Irra	diation		- I		1
Unique ID No.	U grams	U-235 grams	U grams	U-235 grams	U-236 grams	Np-237 grams	Pu grams	Pu-239 grams	Pu-241 grams	Time In Reactor (days)	Cooling Time (days)	Power Level mwd/assy	Burnup %	Decay Heat (Watts) as of: 7/27/93
L-006	236.65	220.11	162.82	134.08	13.34	0.24	0.35	0.27	0.02	1378	2866	2102.50	39.09	2.18
L-007	237.18	220.60	163.21	134.54	13.27	0.24	0.33	0.26	0.02	1367	2784	2094.63	39.01	2.19
L-008	236.76	220.21	162.79	134.04	13.35	0.24	0.34	0.26	0.02	1367	2784	2096.63	39.13	2.20
L-009	236.89	220.33	163.72	135.14	13.18	0.26	0.38	0.28	0.02	1539	2524	2301.78	38.66	2.22
L-010	236.86	220.30	159.99	130.72	13.87	0.27	0.38	0.29	0.02	1743	2320	2364.16	40.66	2.37
L-013	236.80	220.25	161.56	132.58	13.58	0.25	0.34	0.25	0.02	1851	2065	2075.40	39.81	2.41
L-014	237.68	221.07	161.45	132.46	13.59	0.23	0.33	0.25	0.02	1392	2524	2065.87	40.08	2.30
L-015	237.36	220.77	161.50	132.51	13.59	0.25	0.35	0.27	0.02	1456	2163	2135.03	39.98	2.43
L-016	237.23	220.65	162.61	133.83	13.38	0.28	0.38	0.29	0.02	1602	2163	2356.25	39.35	2.37
L-017	237.0	220.43	162.61	133.83	13.38	0.27	0.36	0.28	0.02	1602	2163	2356.25	39.29	2.37
L-018	237.72	221.10	154.98	124.79	14.79	0.26	0.36	0.28	0.02	2426	1493	2317.50	43.56	3.40
L-019	236.75	220.20	162.20	133.34	13.46	0.24	0.35	0.27	0.02	1573	1960	2275.22	39.45	2.49
L-020	236.40	219.88	156.99	127.18	14.41	0.29	0.37	0.29	0.02	2079	1449	2427.20	42.16	3.02
L-021	237.13	220.55	163.10	134.14	13.49	0.18	0.29	0.26	0.02	1463	1960	2118.55	39.18	2.52

#### G. Specifications for Failed/Warped Fuel Units

- Fuel elements or assemblies distorted beyond specified dimensional limits must be considered on an individual basis. The Customer should provide DOE with complete dimensional information for each failed/warped unit at least 120 days before delivery.
- If material normally removed from the element or assembly by the Customer cannot be removed due to fuel failure, warpage, or other reasons, the Customer must notify DOE at least 270 days before delivery giving complete dimensional, material, and weight information. Detailed structural drawings are also required by DOE.

#### H. Cask and Basket Identification

The fuel being received under this Appendix A contract must be identified with a specific cask and basket type.

Cask	Basket	Number of Assemblies/Cask
LWT	(2) 7 Square	14